



PERFORMANCE ASSESSMENTS FOR FUELS AND MATERIALS FOR ADVANCED NUCLEAR REACTORS

Prof. Daniel LaBrier Idaho State University, USA 28 May 2020







































Meet the Presenter



Dr. Daniel LaBrier is an Assistant Professor of Nuclear Engineering at Idaho State University. He earned his doctorate in nuclear science and engineering from ISU in 2013, with an emphasis in irradiated materials characterization. His research focuses on characterizing nuclear-grade materials that are exposed to extreme environments and nuclear reactor safety projects, including investigation of corrosion and erosion of structural materials relevant to LWR and advanced (SFR, MSR, HTR) systems. His research interests include development and qualification of fuels and materials for advanced reactor concepts, investigating thermal hydraulic effects on material performance, and used fuel recycling techniques.



In the recent past, Dr. LaBrier has contributed to projects related to chemical effects testing for Generic Safety Issue (GSI)-191, materials testing capability development for the TREAT reactor restart, and design of advanced reactor testing systems. After serving as a post-doctoral fellow at the University of New Mexico and as a research professor at Oregon State University, Dr. LaBrier returned to ISU in March 2019 and maintains residence as a researcher at the Center for Advanced Energy Studies (CAES) in Idaho Falls, ID.



Email: labrdani@isu.edu

Disclaimer



- Views presented here are solely my own, and:
 - Do not support a particular reactor technology or pathway to deployment,
 - Do not necessarily support a specific entity, whether my home institution (Idaho State University), my funding partners, or any associated affiliates,
 - And are not supported financially by any entity.

Introduction



- A host of novel fuel and material concepts are being investigated as part of the GenIV reactor development initiative.
- While many of these candidates are rooted in historical programs from previous reactor development campaigns, most of these concepts were never fully evaluated for long-term performance in non-LWR facilities.

Introduction (cont.)

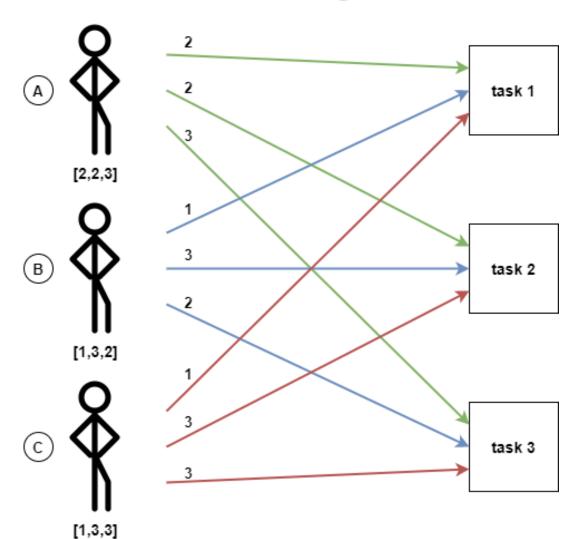


- Performance data that is needed for material downselection, feasibility studies, and eventual qualification is, currently, very costly in terms of monetary cost and human capital.
- The use of an 'all of the above' strategy for performance assessment is needed to reduce the cost of ushering materials through the qualification process.
- In this presentation, we will discuss the efforts that are currently underway, and those planned for the near future, to advance many of these candidates from concept to deployment

Impetus

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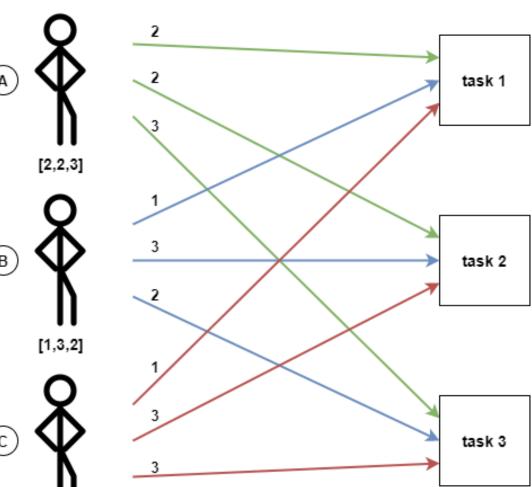
- A variety of technological paths exist for:
 - Core design
 - Core materials
 - Heat exchange system
 - Heat removal system
 - Working fluid(s)



Impetus (cont.)

- When all is said and done, every entity will need to scrutinize their systems through the process of qualification
 - Regardless of technology
 - Regardless of design
 - Regardless of
- All entities are in the same boat…



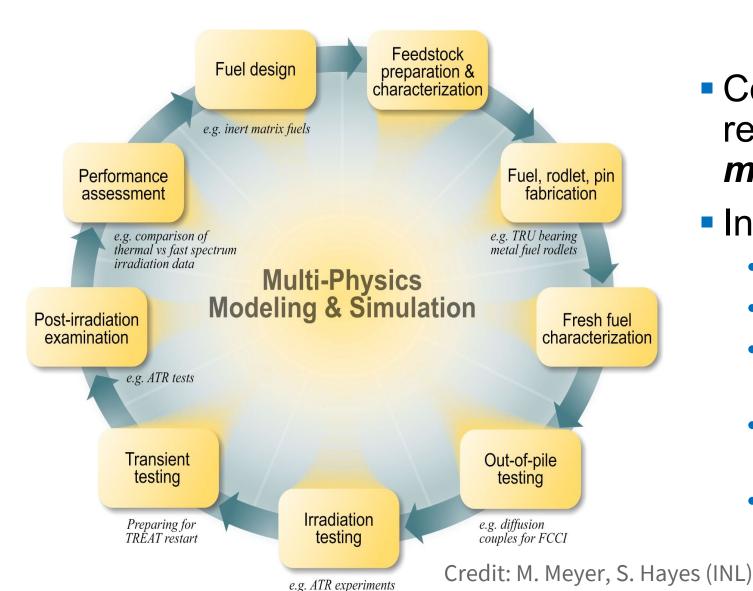




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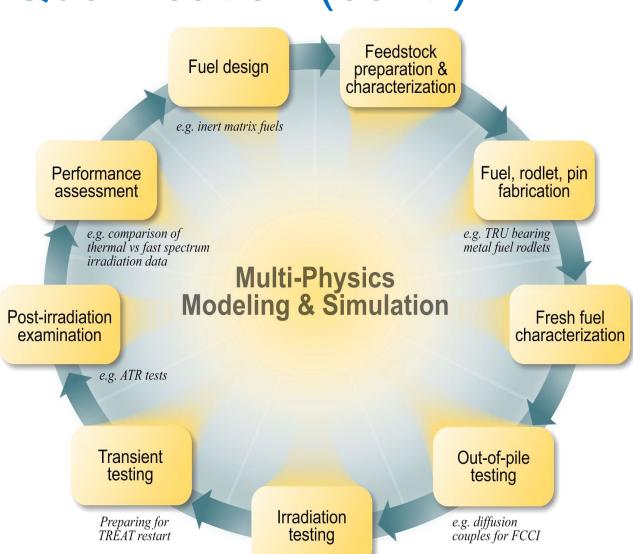
Qualification





- Consider one aspect of reactor development: materials
- Involves every aspect:
 - Fuel and Core
 - Structural
 - Primary and secondary cooling
 - Heat removal and exchange
 - Safety systems

Qualification (cont.)



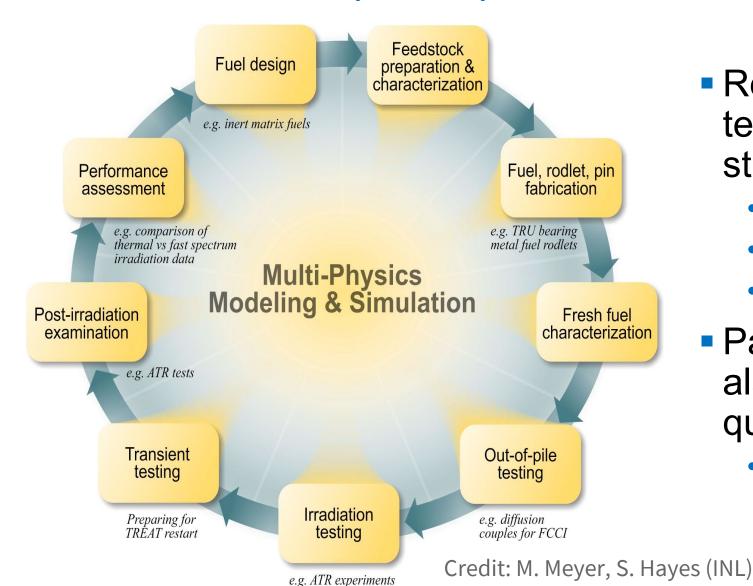
e.g. ATR experiments

Credit: M. Meyer, S. Hayes (INL)



- Modeling and simulation can only go so far...
- Well-vetted experimental data is crucial to successful qualification process

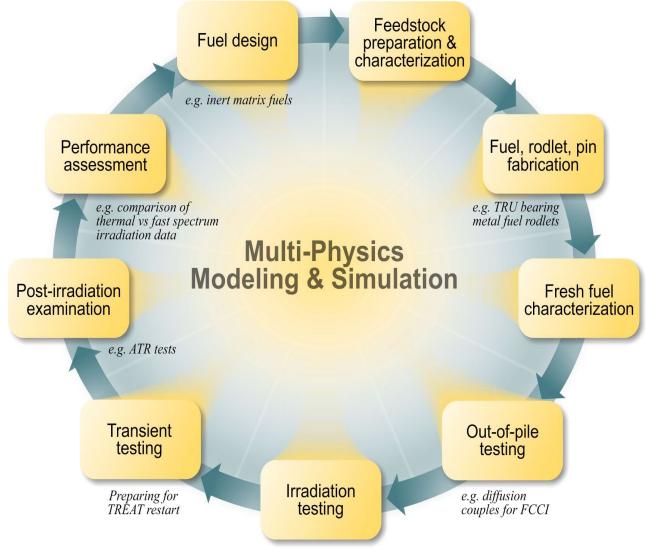
Qualification (cont.)





- Resources for advanced testing are currently stretched thin, especially:
 - Irradiation
 - Safety
 - Transient
- Pathways for qualification also include the need for quality assured work
 - Limits individual vendor testing

Qualification (cont.)



e.g. ATR experiments



- Not very often does the first trial succeed, for ANY technology
- Material testing is an iterative process
- = EXPENSIVE! In terms of money, time, resources

Historical

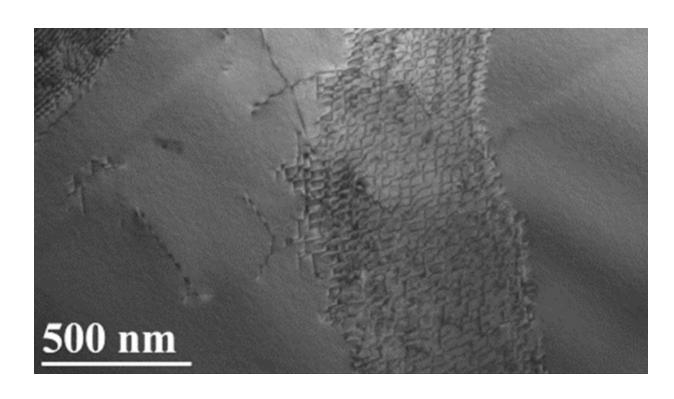


- Process of Fuel and Material Development:
 - Selection of Potential Candidates
 - Lab Scale Development
 - Scale-Up
 - Qualification
 - Demonstration
- Typical schedule for complete development and licensing for new fuel & materials: 20-25 years.

Historical (cont.)



- Recent example: Alloy 617
- Added to ASME Boiler & Vessel Code in May 2020
- Qualification took 12 years,
 \$15M investment from US
 DOE
- First addition to B&PV Code in 30 years

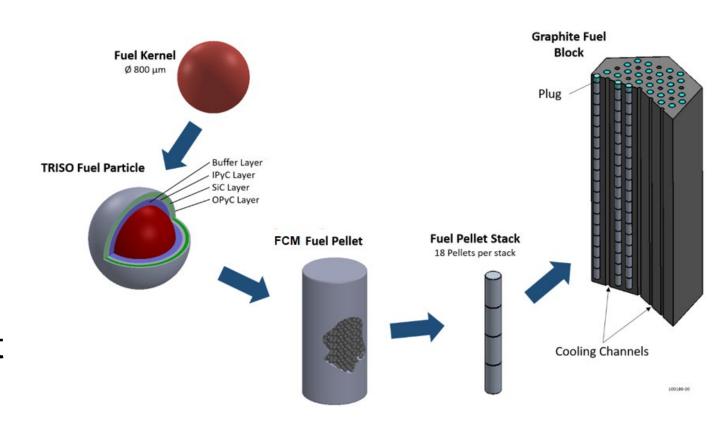


Alloy 617, 500 nm. Credit: INL

Looking to the Future...



- Editorial: if as an industry, we want to deploy advance nuclear reactors, we require:
 - changes in paradigm,
 - thought process,
 - reimaging &/or optimizing all possible uses for data
- "All of the above" strategy is a great sound byte...but what exactly does that mean?

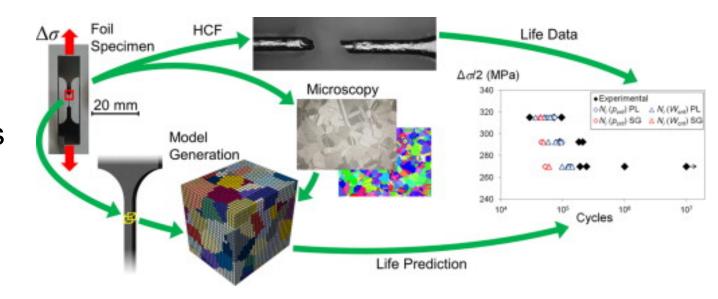


Credit: USNC, 2020

All of the above Strategy

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- How to best approach these challenges?
 - Reduce dose/irradiation hazards
 - Reduce footprint
 - Process automation
 - Develop correlations and models to better predict scale-up processes

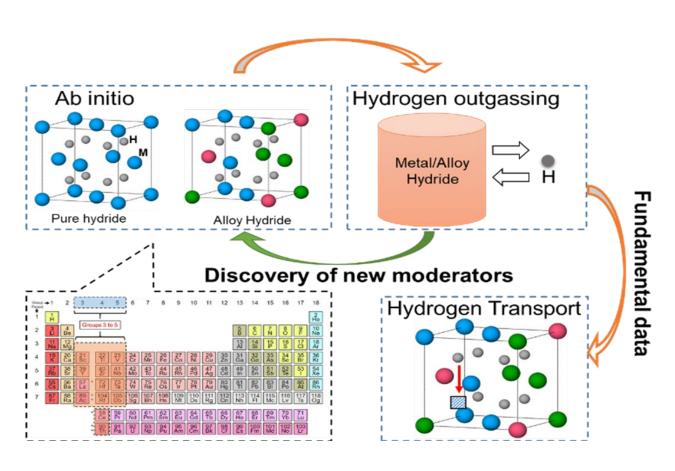


Credit: C.A. Sweeney, 2015

All of the above Strategy



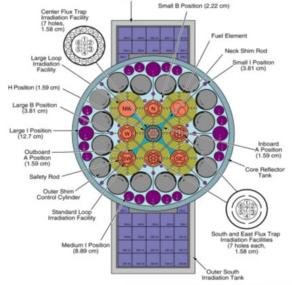
- Design
 - Specific figures of merit
- Development
 - new methods for sussing out novel materials
- Performance
 - More flexible testing methods
 - More testing facilities
- Post-performance assessment
 - More flexible analysis methods
 - More facilities



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- Operations
 - Physical, mechanical
- Irradiation
 - Flux density, neutron spectrum
- Safety
 - DBA or BDBA conditions











Credit: INL, ORNL

Operations

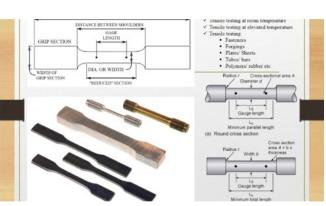
- Physical, mechanical, chemical
- High temperatures, gradients
- Corrosion, erosion
- Oxidation
- Fatigue, creep, stress/strain
- Multiple facilities can be used
- Timelines may be shortened
- However, testing still needs to be sufficient for regulatory concerns







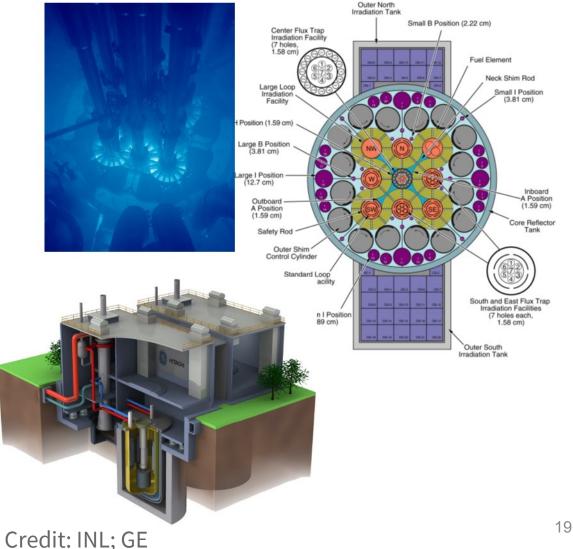




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- Irradiation
 - Flux density
 - Neutron spectrum
 - Fluence
 - Limited number of facilities that can satisfy normal ops conditions, and are in high demand
 - Even more concern for satisfying regulatory concerns

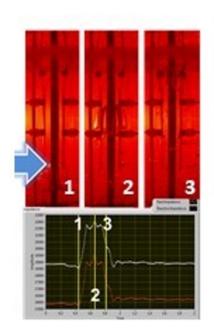


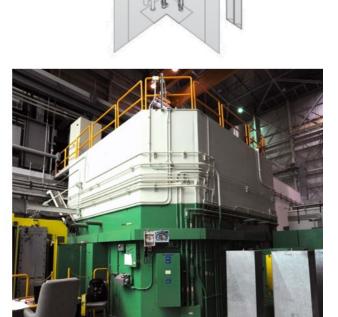


GEXIII International Forum

- Safety
 - DBA or BDBA conditions
 - Can be in-pile or out-of-pile (depending on goal of test)
 - Extremely limited facilities, extremely high demand
 - One limiting factor: amount of damage (dpa) that can be imparted on a sample vs. time

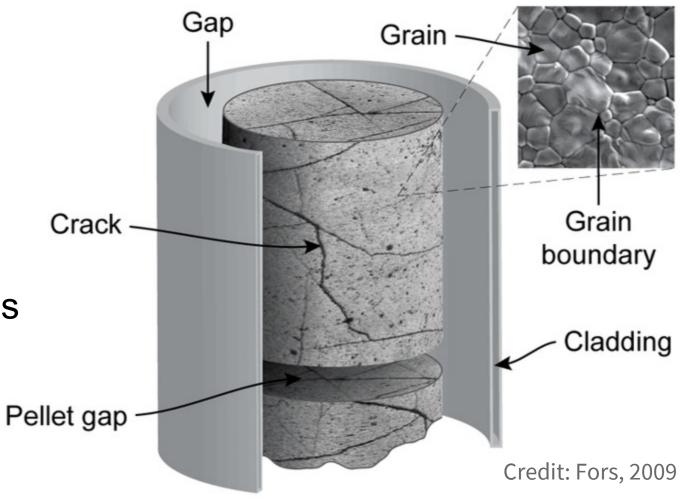






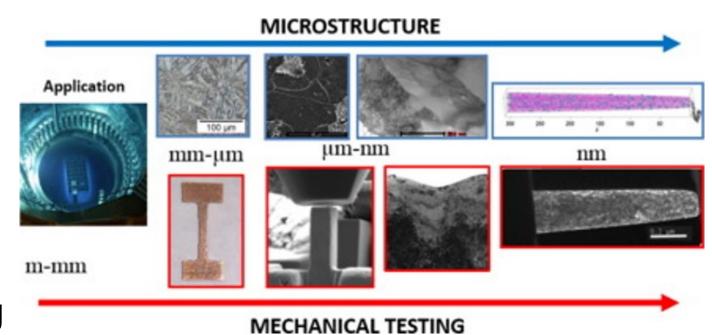
- Reimagine how data can be collected and applied to material qualification
- Traditional use of bulk properties to assess failure mechanisms in materials
- Harder to accomplish with radioactive/irradiated materials





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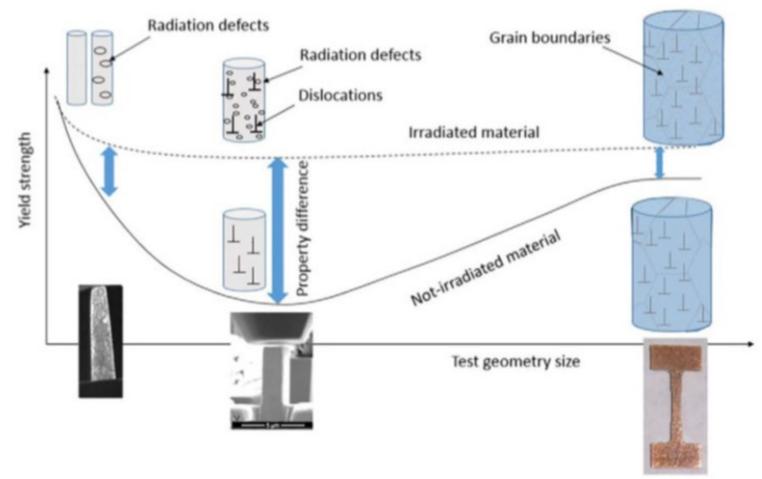
- Better understand connection between microscale and macroscale correlations
- Develop concepts for automated testing systems
- Demonstrate advantages to automated mechanical testing and data management using reduced scale samples



Credit: Hosemann, 2017



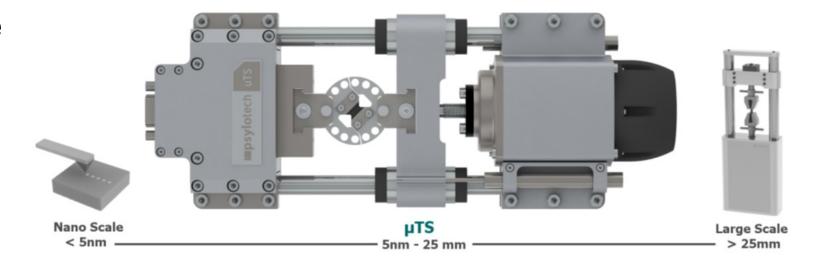
- Micromechanical testing capabilities have improved drastically over the past decade
- The ability to represent bulk property information from microscale sample analysis is a key development!



Credit: Hosemann, 2018



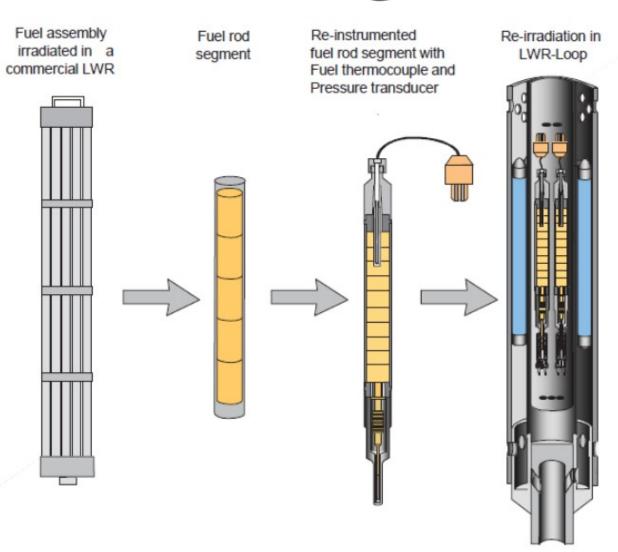
- Achievable using three key principles:
 - Repeatable methods
 - Correlation development
 - Large quantity testing
- Allows for more testing and better testing – relief in the data pipeline



Credit: Psylotech

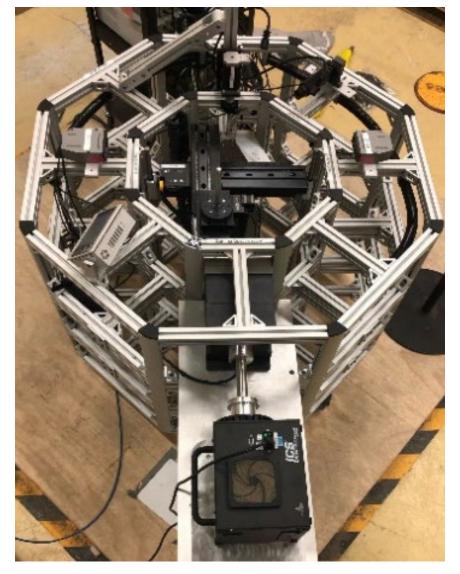
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- Refitting fuel samples for multiple test campaigns
- Performed at Halden facility (Norway) for decades
- Present need for similar testing capabilities



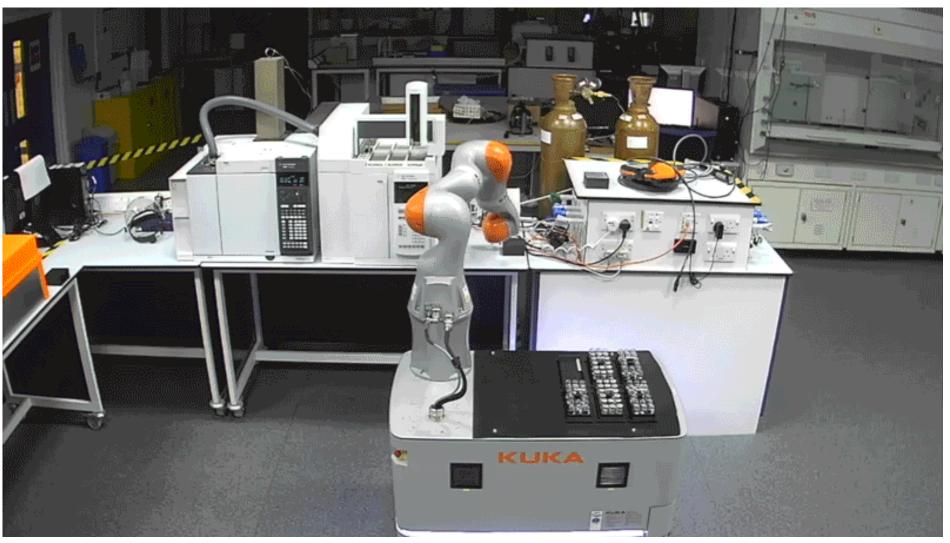
 Multiple PIE techniques fitted to a single device







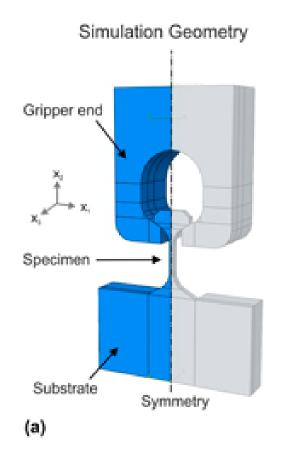
- Use of robotics to improve efficiency
- Suited for areas to minimize dose

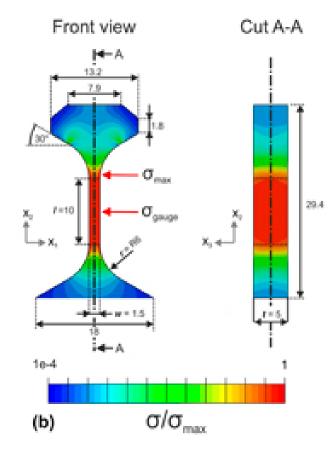


Credit: Univ. Liverpool

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- Use of FIB technology to create more samples (or sample production rate)
- Automation Initiative use of robotics to perform tasks
 - Widely incorporated in the automotive and medical industries
- Reduces timeline on both the front- and back-end of testing



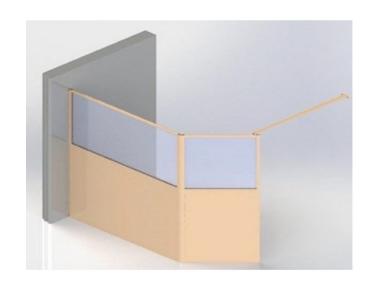


Credit: D. Casari, et al. (2019)

Methods: Reimagined



- Reduced size of samples for testing results in lesser dose
- Tasks that often require monolithic hot cells might be accomplished in a laboratory setting
- Use of modular shielding to reduce dose locally







Role of Quality Assurance

Editorial: if any entity were considering the development and implementation of an NQA-1 compliant research program, this would be a great time to start!



ASME NOA-1-2017 **Quality Assurance** Requirements for **Nuclear Facility** Applications AN AMERICAN NATIONAL STANDARD Copyrighted Material

Credit: ASME 2017

Mechanisms



- Getting access to more facilities is paramount to success so who can be called upon?
- NSUF Nuclear Science User Facilities
- GAIN Gateway for Accelerated Innovation in Nuclear
- University Collaborators
- Industrial Collaborators

Summary



- Variety of novel reactor types under development is a blessing for the nuclear community...but
- Strains the available resources for assessment, irradiation and safety testing, qualification development
- Drives innovative thinking on how to assess materials for new reactor types

Thank you!









Upcoming Webinars

24 June 2020 Comparison of 16 Reactors Neutronic Dr. Jiri Krepel, PSI, Switzerland

Performance in Closed Th-U and U-Pu Cycles

29 July 2020 Overview of Small Modular Reactor Technology Dr. Frederik Reitsma, IAEA

Development

26 August 2020 Overview and Status Update on Molten Salt Dr. David Holcomb, ORNL, USA

Reactor Technology Development in the US